

## LA-UR-13-28503

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Title: Comparison of ENDF71x and ENDF70 Using ICSBEP Criticality Benchmarks in MCNP6

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Intended for: 2013 ANS Winter Meeting and Nuclear Technology Expo,  
2013-11-10/2013-11-14 (Washington, District Of Columbia, United States)

Issued: 2013-11-05



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# Comparison of ENDF71x and ENDF70 Using ICSBEP Criticality Benchmarks in MCNP6

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October 29, 2013

# Introduction

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## ENDF/B-VII.1

### Evaluated Nuclear Data File Version 7.1

- Released December 2011
- Neutron sublibrary:
  - 423 evaluations, 32 new in ENDF/B-VII.1
  - Elemental evaluations for V and Zn became isotopic evaluations:  
 $^{50}\text{V}$ ,  $^{51}\text{V}$  and  $^{64}\text{Zn}$ ,  $^{65}\text{Zn}$ ,  $^{66}\text{Zn}$ ,  $^{67}\text{Zn}$ ,  $^{68}\text{Zn}$
  - 10 excited-state evaluations, 413 ground-state evaluations

## ENDF71x

423 ENDF/B-VII.1-based ACE data tables

- 423 evaluations, 32 new
- Best neutron XS data available
- Available with MCNP6 from RSICC
- *Extensive* validation and verification

Temperature	ENDF71x	ENDF70
293.6 K	80c	70c
600 K	81c	71c
900 K	82c	72c
1200 K	83c	73c
2500 K	84c	74c
0.1 K	85c	
250 K	86c	

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# Validation using ICSBEP Benchmarks

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## ICSBEP

### *International Handbook of Evaluated Criticality Safety Benchmark Experiments*

- 717 MCNP6 models (Kahler and Mosteller)
  - 18 “classes” of benchmarks
- 500 to 5000 active cycles
- 10,000 histories per cycle
- 6851 CPU hours

# ICSBEP Classes of Benchmarks

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**FUEL** Primary fuel in system

**HEU** Highly-enriched uranium

**IEU** Intermediate-enriched uranium

**LEU** Low-enriched uranium

**MIX** Mix of  $^{235}\text{U}$  and  $^{239}\text{Pu}$

**PU**  $^{239}\text{Pu}$

**U233**  $^{233}\text{U}$

**CHEM** Chemical composition

**MET** Metal

**COMP** Compound

**SOL** Solution

**MISC** Other

**SPEC** Flux spectrum

**FAST** 50 % of  $\psi > 100 \text{ keV}$

**INTER** 50 % of  $\psi > 0.625 \text{ eV}$   
 $< \psi < 100 \text{ keV}$

**THERM** 50 % of  $\psi > 0.625 \text{ eV}$

**MIX** All other spectra

**EVAL** Number identifying experiment

**CASE** Number identifying experiment configuration

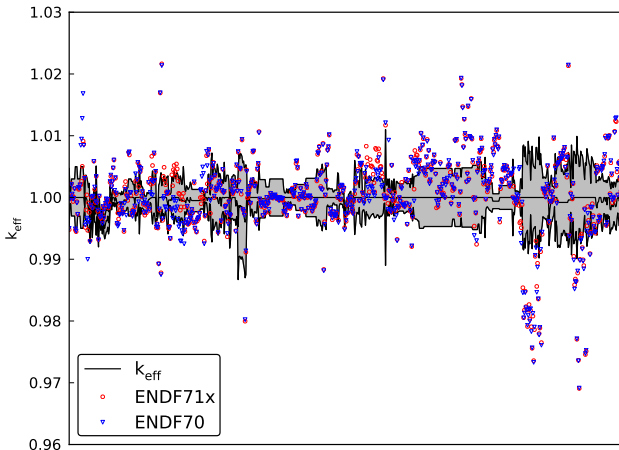
**Example**

HEU-MET-FAST-001-001

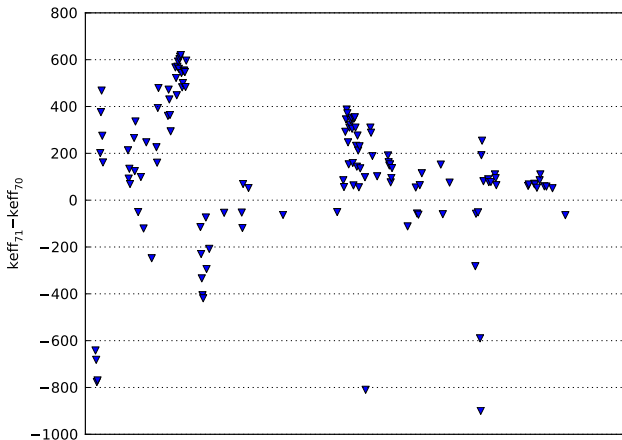
# ICSBEP Classes of Benchmarks

Benchmark Group	#	Benchmark Group	#
HEU-COMP-INTER	1	MIX-COMP-THERM	6
HEU-MET-FAST	174	MIX-MET-FAST	34
HEU-MET-INTER	4	PU-COMP-INTER	1
HEU-MET-MIXED	5	PU-MET-FAST	43
HEU-SOL-THERM	46	PU-SOL-THERM	122
IEU-COMP-THERM	1	U233-COMP-THERM	2
IEU-MET-FAST	16	U233-MET-FAST	10
LEU-COMP-THERM	101	U233-SOL-INTER	26
LEU-SOL-THERM	22	U233-SOL-THERM	103

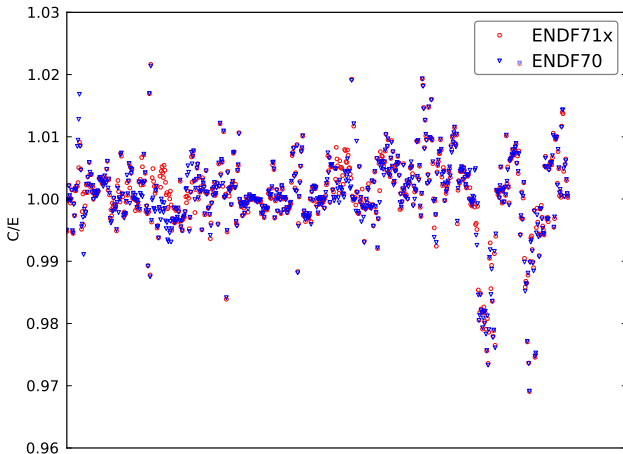
# MCNP6 $k_{\text{eff}}$ Results



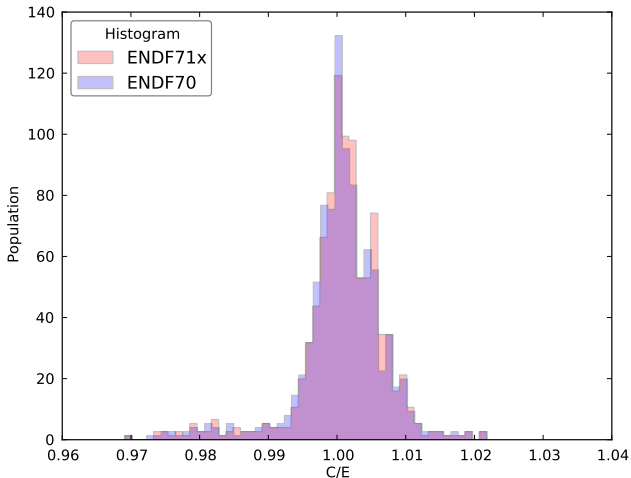
# MCNP6 $k_{\text{eff}}$ Results



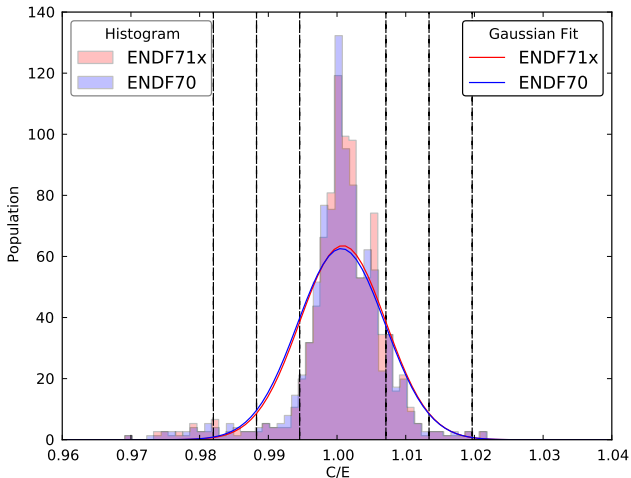
# MCNP6 $k_{\text{eff}}$ Results



# Statistical Deviations

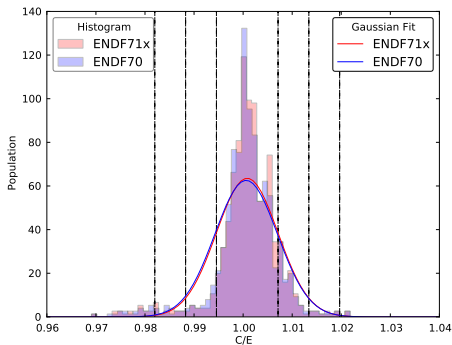


# Statistical Deviations



# Statistical Deviations

## Attempting a Gaussian Distribution



How many models fall within  $x$  standard deviations  $\sigma$ :

	%	Exp.	71x	70
$1\sigma$	68.3	488	335	338
$2\sigma$	95.4	682	548	550
$3\sigma$	99.7	713	642	636
$4\sigma$	99.9	714	676	674

# Biggest Players

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## Largest $\Delta k_{\text{eff}}$

1. u233-met-fast-004-002
2. pu-met-fast-005-001
3. heu-met-fast-003-010
4. heu-met-fast-003-011
5. heu-met-fast-003-009
6. heu-met-fast-003-008
7. heu-met-fast-066-009
8. heu-met-fast-066-008
9. heu-met-fast-066-007
10. heu-met-fast-077-008

## Partial reasons for change

1. Beryllium
2. Tungsten
3. Titanium

# Biggest Players

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## Furthest from experiment

1. heu-met-fast-033-002
2. heu-met-fast-033-001
3. heu-met-fast-004-001
4. heu-met-fast-057-005
5. pu-sol-therm-028-006
6. pu-sol-therm-028-005
7. pu-sol-therm-028-009
8. pu-sol-therm-028-003
9. heu-met-inter-006-004
10. mix-met-fast-008-007

# Conclusion

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- ENDF71x library released with MCNP6
- Best neutron cross section data available
- Extensive validation:
  - 717 ICSBEP MCNP6 benchmark models
  - 548 fall within  $2\sigma$ , expected 682
- Majority of changes may be attributed to:
  - Beryllium
  - Titanium
  - Tungsten
- More work to find cause for large discrepancies